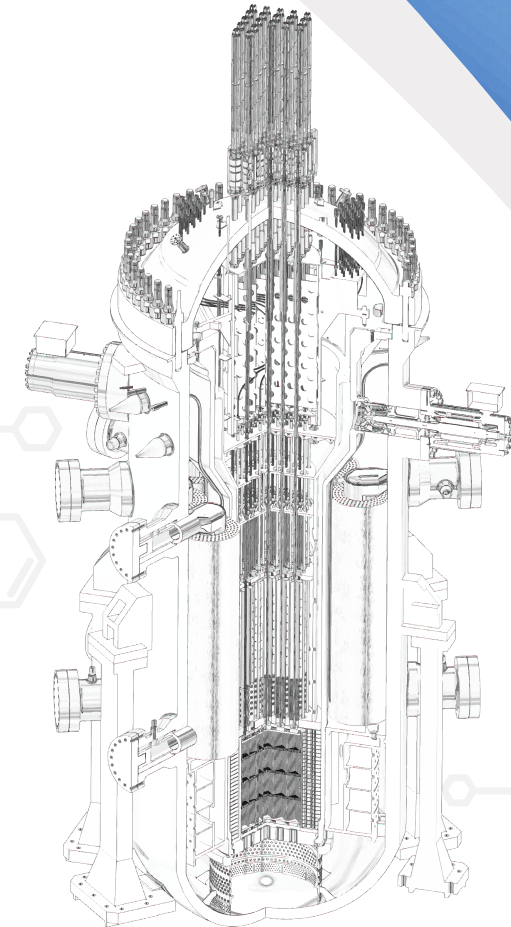


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# SMART100 Integral Reactor and Other SMRs under Development in Korea

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- 01** Korean Nuclear Power Industry
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- 03** Non-Water Cooled SMRs
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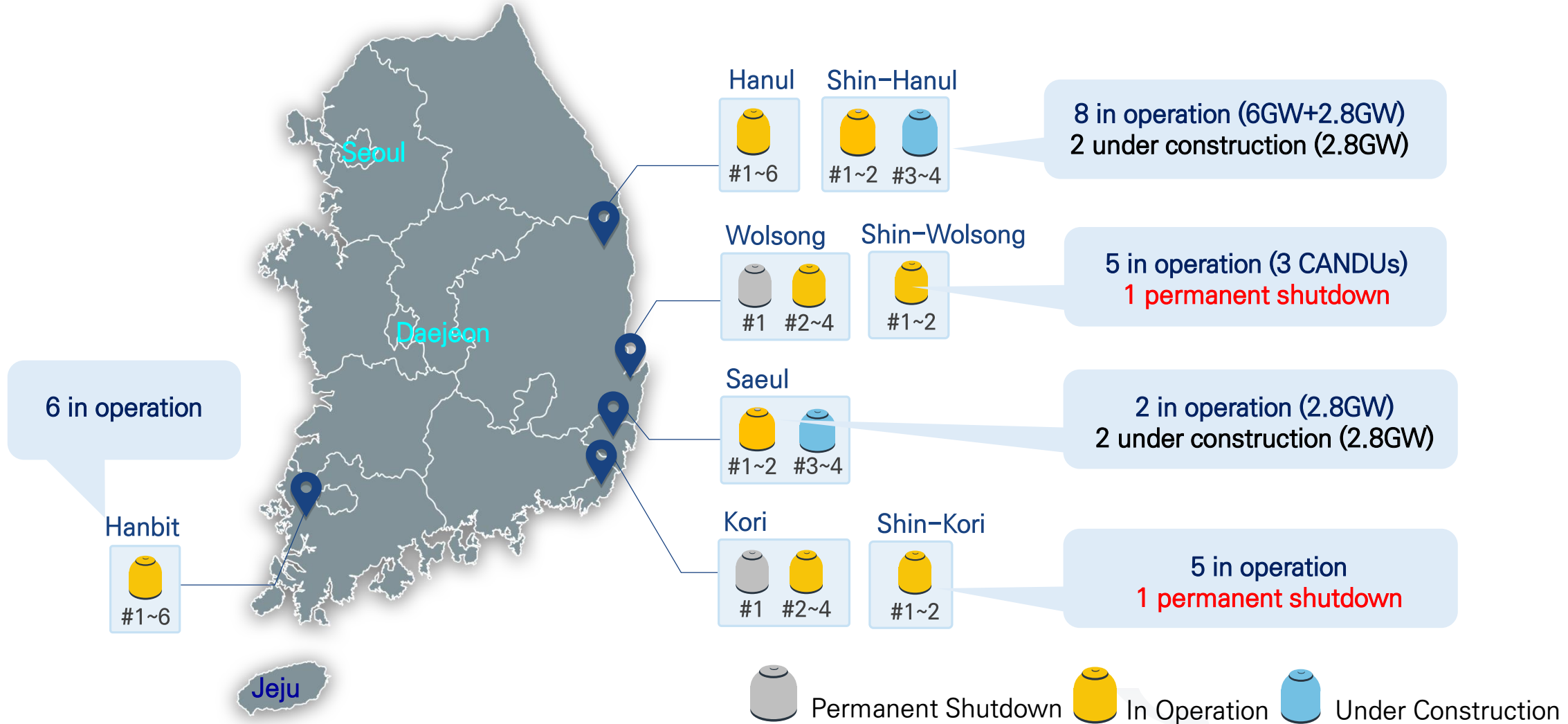


# Status of Nuclear Power Plants in Korea

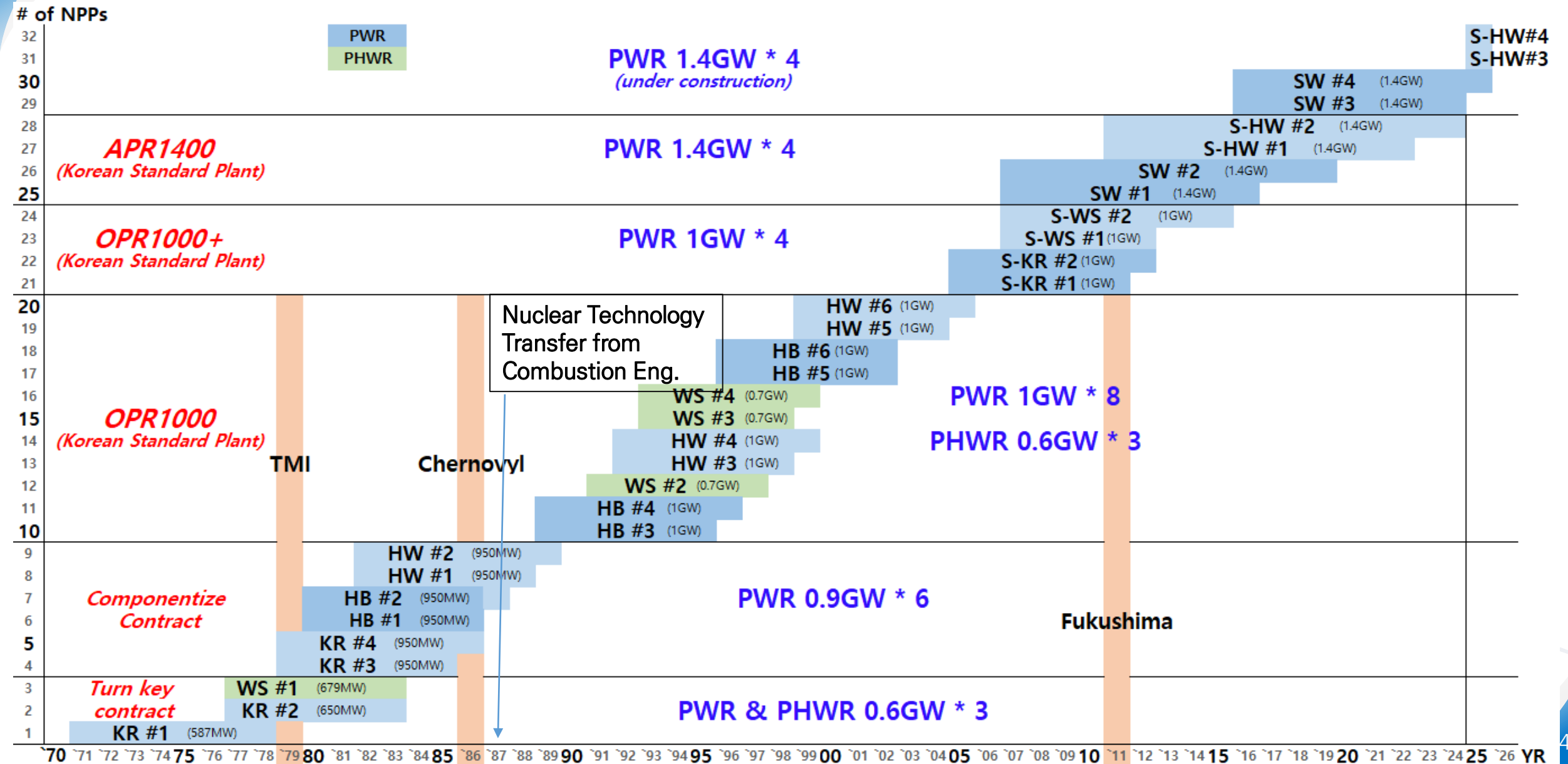
26 Units (26GW, 4 APR1400s) In Operation

4 Units (4 APR1400s) Under Construction

4 Units (1 SMR) Planned



# Nuclear Power Plant Construction History



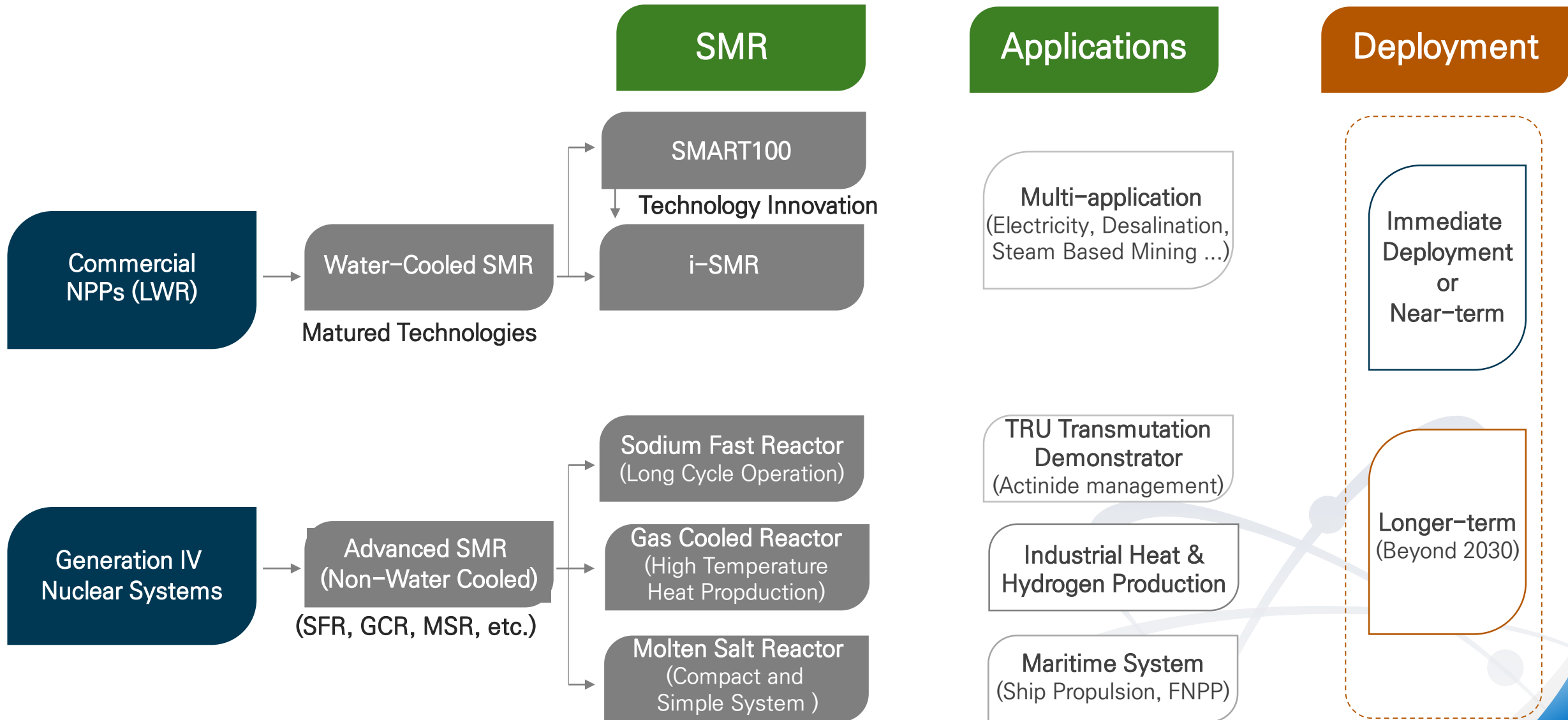


# Nuclear Industry Infra Structure

Self-sustaining Nuclear Power Eco System with Full Technology Independence



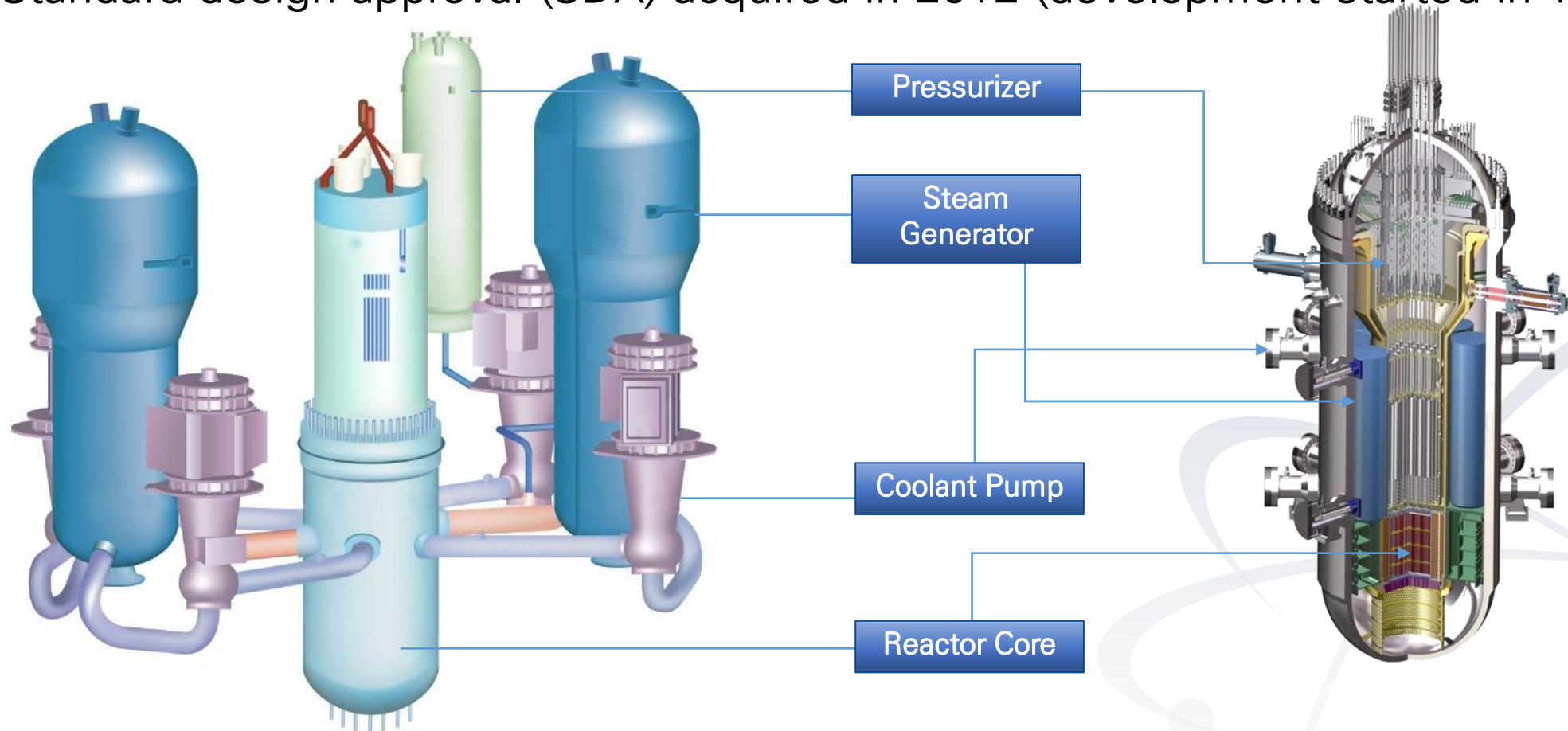
# Advanced Reactor Development Program of Korea



# SMART (System-integrated Modular Advanced Reactor)

## Integral Reactor with 330 MWth and 100 MWe Suitable for Small Grids

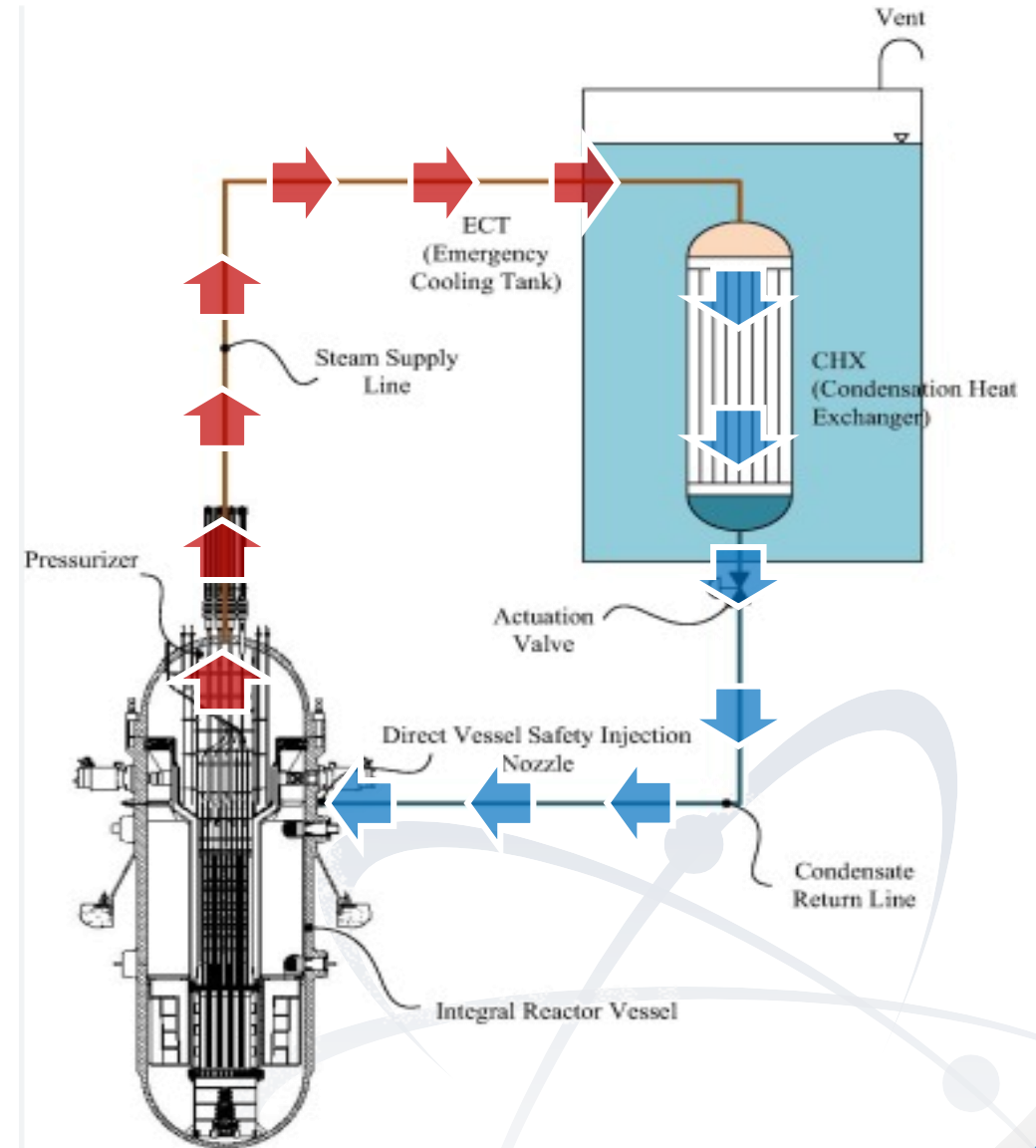
- Integration of major components into a vessel to prevent potential leak
- Standard design approval (SDA) acquired in 2012 (development started in 1997)





# SMART100: Enhanced Version of SMART

- **Enhanced Passive Safety**
  - Passive heat removal by natural circulation in accident conditions
- **Collaboration with Saudi Arabia**
  - Pre-project engineering started in 2016
- **Versatile Use with Increase Capacity**
  - Electricity Generation and Desalination
    - 100MWe and fresh water of 40,000 ton/day with 365 MWth
  - Steam supply for mining
- **Immediately deployable SMR**
  - SDA obtained in Sept. 2024
  - Manufacturability confirmed by Doosan

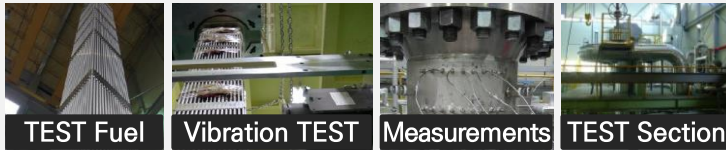


# Comprehensive SMART Technology Validation Experiments

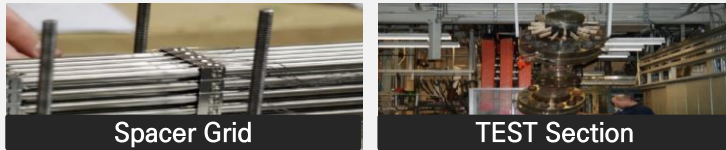
## to Verify Systems, Components and Design Tools for Licensing

### Fuel Thermal-Hydraulics Tests

#### > Fuel Performance Tests



#### > CHF Measurement Test

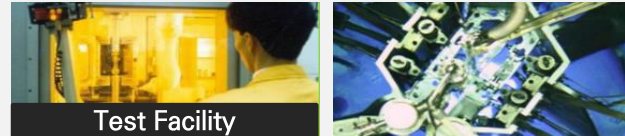


### Mechanics and Components

#### > RPV Dynamics Test, RCP Mockup Test and Helical ISI Test



#### > SG Tube Material (A690) Irradiation Test



### System Thermal-Hydraulics Experiment



### SMART – Integral Test Loop

#### > World's Unique and Largest Full Scope Accident Simulation

1:1 Height, 1/49 Volume



### SMART – Main Control Room Simulator



RPV : Reactor Pressure Vessel

RCP : Reactor Coolant Pump

ISI : In-Service Inspection

PRHRS: Passive Residual Heat  
Removal System

VISTA: Experimental Verification by  
Integral Simulation of Transient  
and Accident

ITL : Integral Test Loop

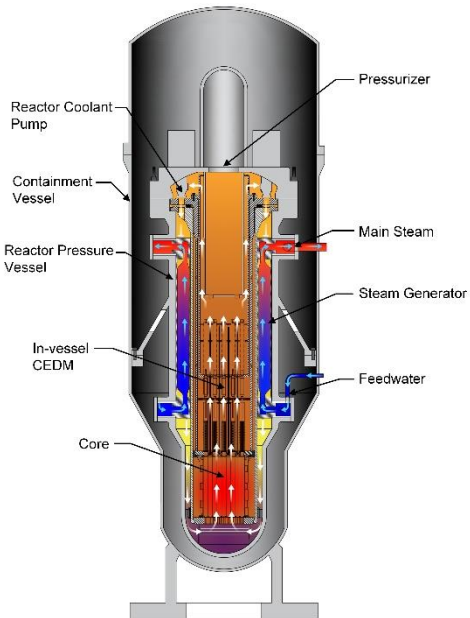
MCR: Main Control Room

# i-SMR to Enhance Safety Further as well as Economics

» Innovative Technology and System Design to Achieve the Top-Tier Requirement for Safety (CDF\* ~  $10^{-9}$ ) and Economic (3500\$/kWe) Goals I

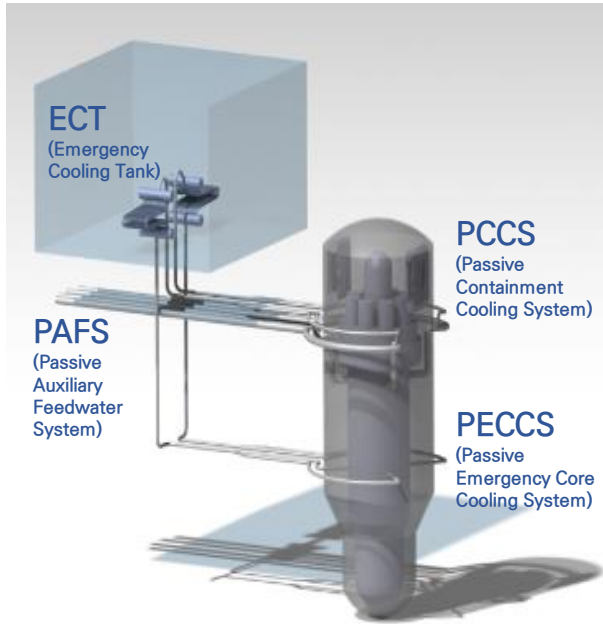
## Primary System

- » Monoblock Steam Generator
- » Steel Containment Vessel to Retain Potential Leakage Water



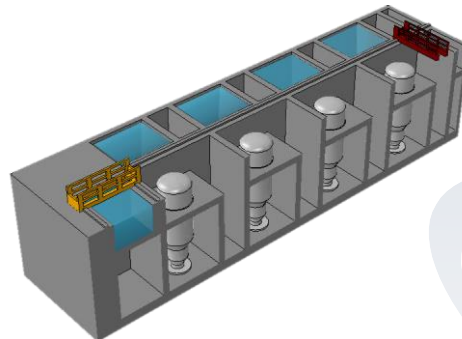
## Safety System

- » Indefinite, Unattended, Unpowered Safety System
- » Whole Covered System from AOO to Severe Accident



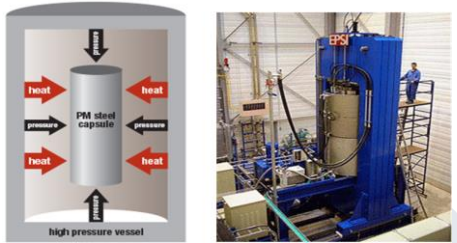
## 4 Module System

- » Bigger Module of 170 MWe
- » 4 Modules to 680MWe  
– Adequate for Coal Power Replacement

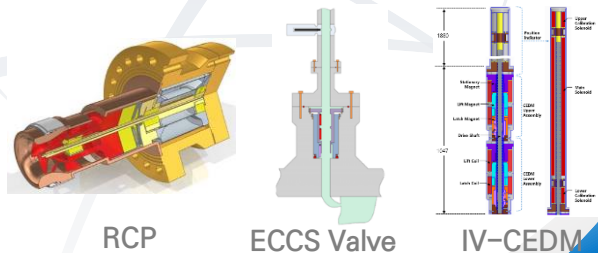


## Innovative Tech.

- » Innovative Manufacturing Tech to Reduce Construction Period



- » Three Major Components:  
– Pump, Valve, In-Vessel CEDM

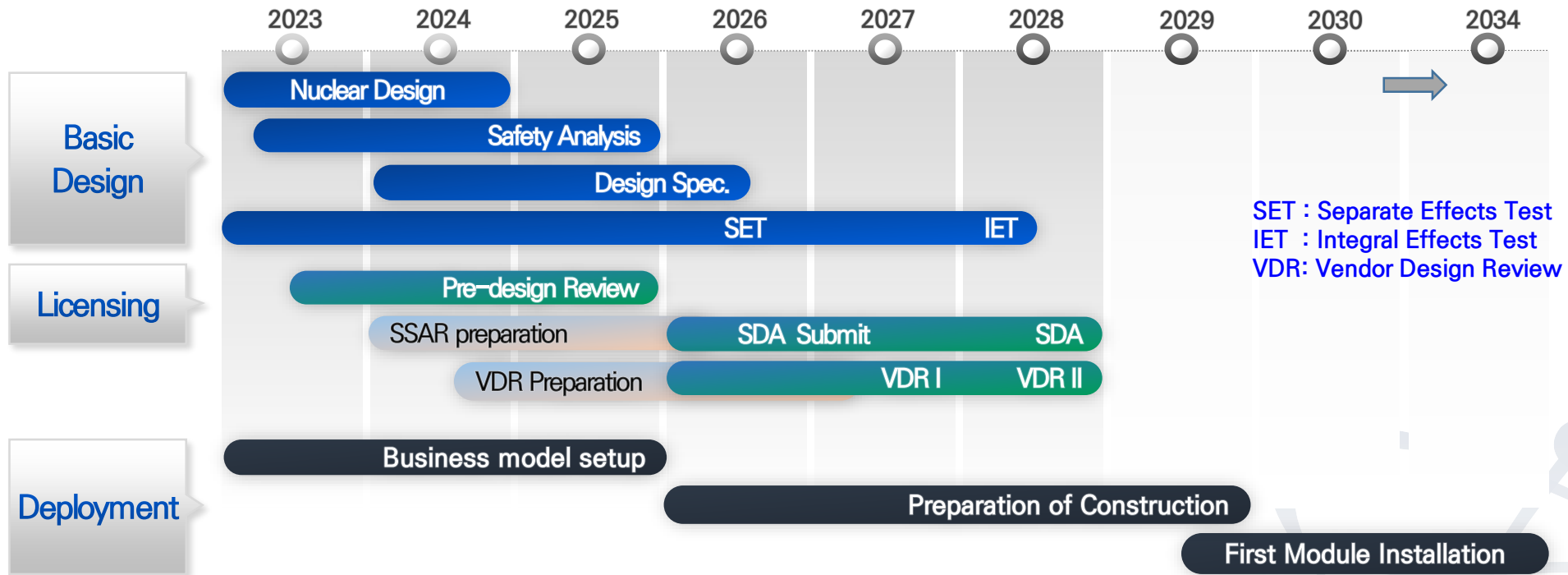


\* CDF : Core Damage Frequency



# i-SMR Development and Construction Plan

- Government funding has been started for standard design program in 2023.
- SDA acquisition by 2028 and first module installation by not later than 2034

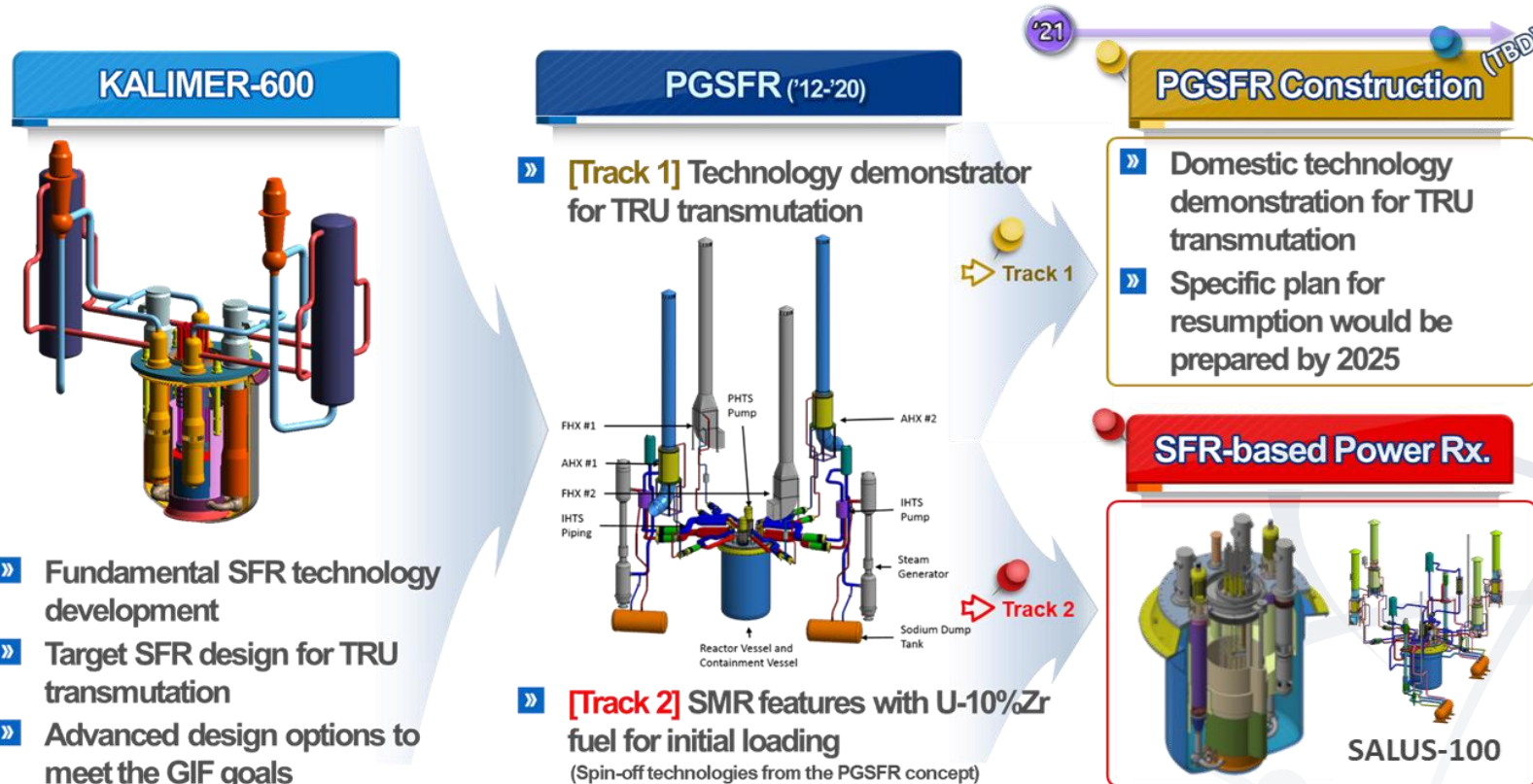


# Sodium-cooled Fast Reactor

## Development History

- Conceptual design (2007) of KALIMER-600 (Pool-type, Metal fuel, Passive safety features, etc.)
- Engineering design (2020) of PGSFR, TRU-transmutation demonstrator with pyroprocessing
- Basic design of SFR-based SMR with a long fuel cycle core (SALUS-100), in progress

PGSFR: Prototype Generation-IV Sodium-cooled Fast Reactor  
SALUS: Small, Advanced, Long-cycled and Ultimate Safe SFR



# SALUS-100 and Future SFR Development Plan

## General Features of SALUS-100

- Spin-off from the PGSFR Development
  - Based Prototype Gen IV SFR which was for transmutation tied with pyro-processing halted in 2022
  - Longer cycle (20year) with lower power density
- Integrated pool-type SFR with Metallic alloy fuel (U-10%Zr) & FC92 cladding
- Electric output: 100 MWe (267 MWt) with core Inlet/Outlet Temps. of 360/510 °C
- Enhanced safety features for a long-term cooling capability with Active and Passive DHRS

## Progress

DHRS: Decay Heat Removal System

- Technical Safety Review by IAEA on-going
- Public-Private Partnership Project for SFR-based advanced SMR development
  - Scheduled to begin in 2025 based on a matching fund system with the private company
- HDEC signed an MOU with KAERI dedicated to SFR development  
HDEC: Hyundai Engineering and Construction
- Planning a new project of comprehensive advanced nuclear reactors development encompassing all from design and validation to demonstration



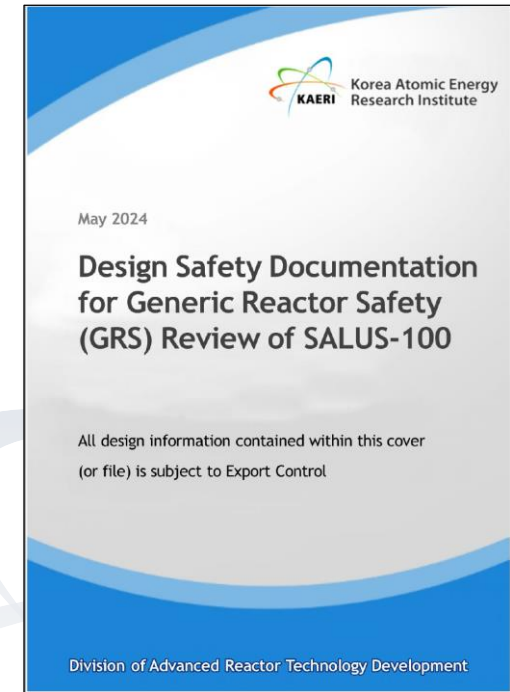


# Technical Safety Review of SALUS-100 by IAEA

## Specific Achievements

\* Technical Safety Review

- Completion of IAEA TSR\*–DS (Design Safety) in 2024
  - Assessment of SALUS-100 design to identify key regulatory issues earlier to avoid or minimize risks that the developer may encounter during the actual licensing phase
  - Generic safety review service to support NPP deployment based on the IAEA safety standards
- TSR Review area of SALUS-100
  - Compliance of international requirements and standards
  - Safety concerns and Potential issue for licensing
  - Adequacy to undertake pre-licensing review
- Key Milestones
  - IAEA TSR Preparatory Meeting (26-Feb-2024)
  - TSR Design Review and Technical Discussion (07-Oct-2024)
  - Official draft of the IAEA TSR report (01-Nov-2024)
    - Final TSR report scheduled for publication in February 2025

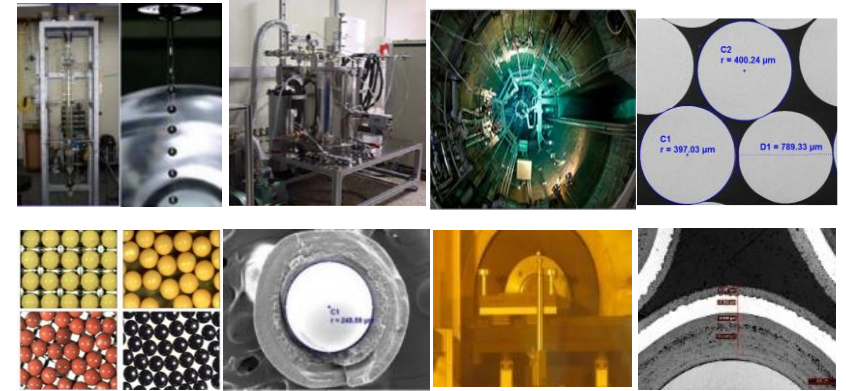


TSR document set of SALUS-100

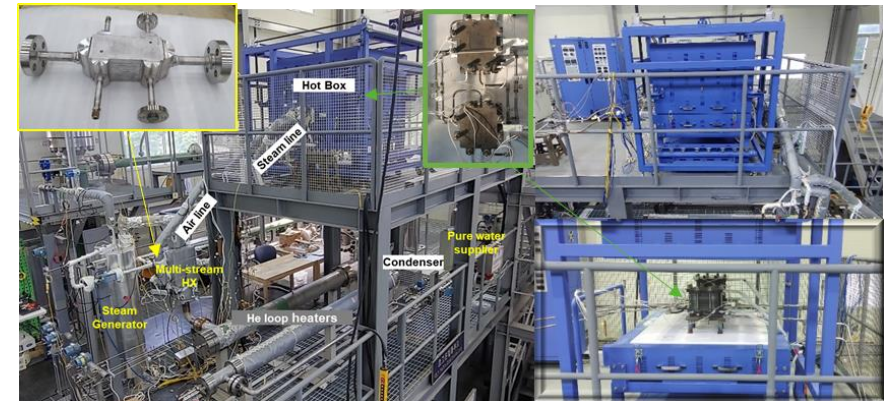
# HTGR for Industrial Heat Supply and Hydrogen Production

## Major R&D Products for GCR Development ('04~'19)

- Computational Tools and Modeling Technologies
  - Neutronics (DeCART/CAPP) and Core T/H Code (CORONA), Safety analysis (GAMMA) and TRISO (COPA) developed
  - Used in the design analysis of MMR of USNC
- Development of TRISO Fuel and High-temperature materials
  - Completion of HANARO Irradiation Test
  - Graphite, Ni-base alloy for Gen IV
- Helium Loop
  - Compact heat exchanger test above 900°C
  - Coupled HTSE (2Nm<sup>3</sup>/hr, 6kWe) tests



〈 TRISO Fuel Manufacturing Technology 〉



〈 Helium Loop and HTSE 〉

# High Temperature Gas-cooled Reactor

- **Overview**
  - Development of essential technologies since 2004
  - Collaborative study with end-users
    - Nuclear HTSE MOU, Alliance of nuclear heat utilization
  - Public-Private Partnership project for HTGR system development since 2024

- **Target Plant**
  - 90 MWth, UCO-TRISO, Graphite moderated, Helium coolant
  - Core outlet temperature: 750°C
  - Non-electric applications for process heat

- **Specific Characteristics**
  - Inherent safety
  - Alternative industrial heat source to fossil fuel





# GCR Public-Private Partnership Participants & Roadmap



Core Design / System Design / Safety Analysis  
Fuel Design / Core Structure Design



Project Management / Plant Design (BOP)



Electrical System Design for Plant



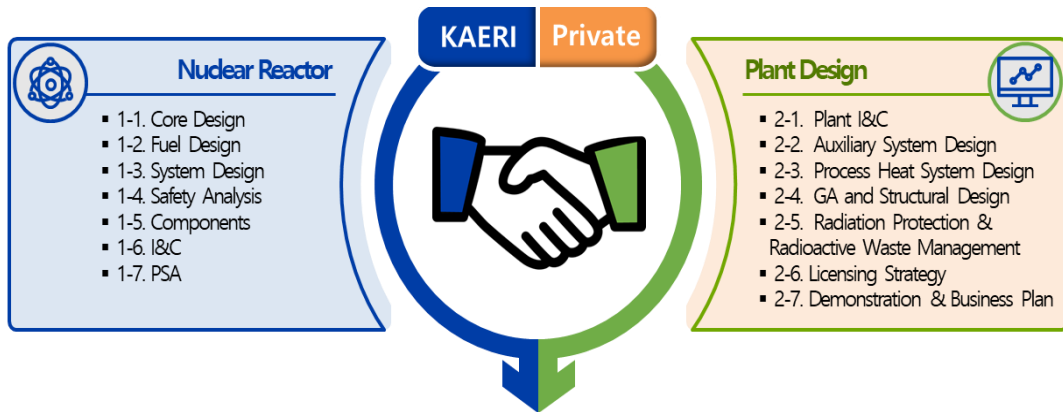
Radiation Protection System Design  
Radioactive Waste Management System



SOEC System Design / Hydrogen Business Plan



Process Heat System Design/ Process Heat Business Plan



PPP Project for Basic Design of HTGR & Process Heat Plant

| Yr                    | ~'24                        | '24                                       | '25 | '26                                  | '27                   | '30 ~  | '35~                      |
|-----------------------|-----------------------------|---|-----|--------------------------------------|-----------------------|--|---------------------------|
| Classification        | Government R&D              | PPP Development Project                   |     |                                      | Demonstration Project |  | Commercialization Project |
| Leading Organizations | Government                  | Government 50                             |     |                                      | Private Sector 50     |  | Private Sector            |
| Project Details       | Development of Technologies | (1 <sup>st</sup> Phase) Conceptual Design |     | (2 <sup>nd</sup> Phase) Basic Design |                       | • PSAR · EIA, FSAR<br>• Site Selection · CP · OP | Business                  |

# URECA – Molten Salt Reactor for Ship Propulsion

## Advantages of Molten Salt Reactor

- Small and simpler structure due to fuel mixed with coolant salt
- No significant dispersion of radioactive materials due to solidification after leak in an accident condition

## Technical Difficulties with Corrosion

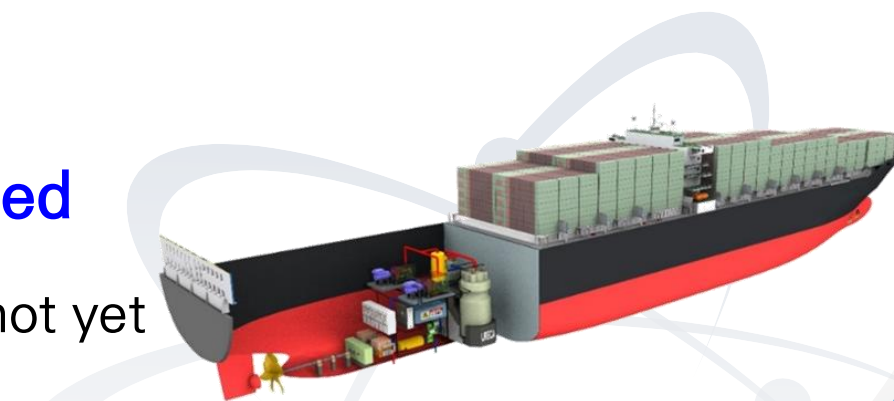
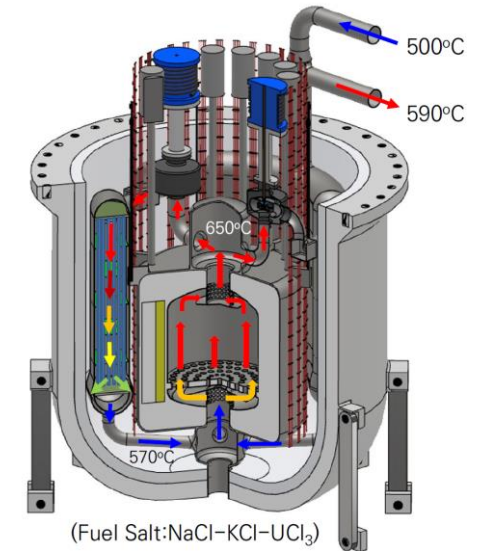
- Cladding with corrosion resistant material layer
- Design with replaceable components

## Small Scale R&D Going on after Selection as a National Innovative Challenge Project in 2022

- Comprehensive demonstration of key technologies by 2026
  - 3.5 year budget of ~29 M\$
- Basic design and license application by 2035
- FOAK for ship propulsion by 2040

## Experimental Facilities in Gampo Site Being Planned

- MOU signed with Seaborg
- Specific plan to build molten salt experimental facilities not yet fixed



# Feature of URECA and Partnership

## Features of URECA

- Thermal Output: ~ 100 MWth
  - 15,000 TEU Container Ship Engine
- Outlet Temperature: over 600 °C
  - Electricity, Hydrogen and Heat Production
- Fuel: NaCl-KCl-UCI<sub>3</sub>
- Neutron Spectrum: Fast

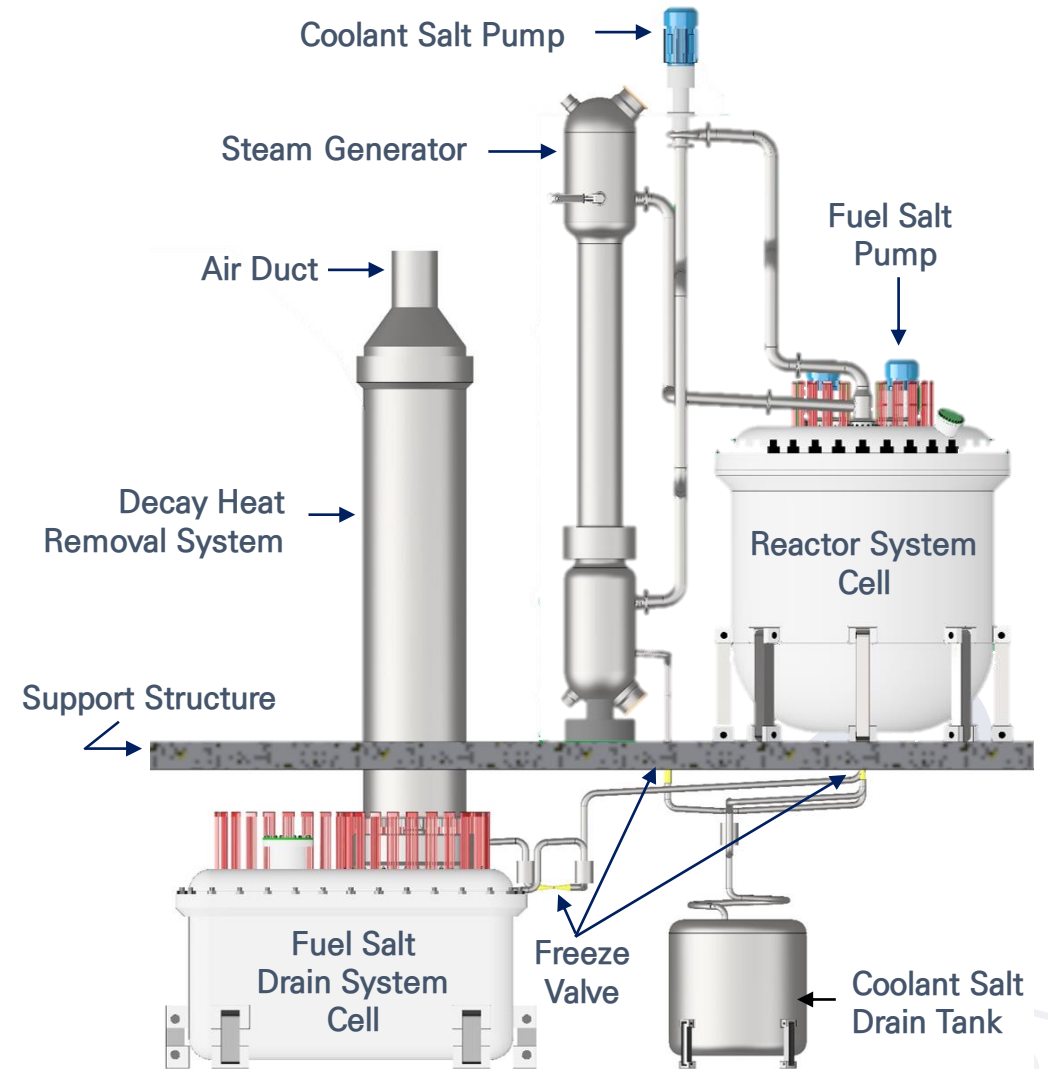
## MSR Partnership



**KIMS** Korea Institute of Materials Science  
Materials and Manufacturing

**INL** Idaho National Laboratory  
Thermophysical Properties of Molten Salt

“Supply Chain Map: domestic and international suppliers”



# Concluding Remarks

## Nuclear Power Technology Well Established in Korea

- Continued construction of 32 nuclear power plants for more than 50 years provided the self-sufficient complete nuclear power plant (NPP) supply system
- It became the base of efficient and low cost NPP construction as proved by the UAE Barakha project
- With the R&Ds of various new reactors being paralleled by construction, the nuclear power technology is well established in Korea

## Various Advanced Reactors Under Development for Carbon Neutrality

- SMART for small electrical grids and resource mining in remotes sites
- iSMR primarily to replace coal power plants
- SALUS for long term operation in remote places
- HTGR for high temperature industrial heat supply and hydrogen generation
- MSR for ship propulsion



We develop Nuclear Technologies for a Better World

